



# An Efficient Separation Method for Terbium and Gadolinium

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## Introduction

Terbium-based radiopharmaceuticals currently show good potential within the field of theragnostics [1]. Specifically,  $^{152}\text{Tb}$  ( $t_{1/2} = 17.4$  h, 20%  $\beta^+$ , ec decay) and  $^{155}\text{Tb}$  ( $t_{1/2} = 5.34$  d, ec decay) for diagnosis and  $^{161}\text{Tb}$  ( $t_{1/2} = 6.89$  d,  $\beta^-$  decay) and  $^{149}\text{Tb}$  ( $t_{1/2} = 4.12$  h, 17%  $\alpha$ , ec,  $\beta^+$  decay) for therapy are sought for their ability as a “theragnostic quartet.” To produce pharmaceuticals containing these radiometals, an efficient separation and isolation process is necessary for Tb. Pure, no-carrier added  $^{161}\text{Tb}$  is produced through the  $^{160}\text{Gd}(n,\gamma)^{161}\text{Gd}(\beta^-)^{161}\text{Tb}$  reaction in nuclear fission reactors. Thus, separating terbium from massive gadolinium targets is currently a barrier to producing high specific activity terbium pharmaceuticals. This abstract proposes a separation method using LN2 extraction chromatography (EXC) resin, which has been tailored for adjacent lanthanide separation.

## Results

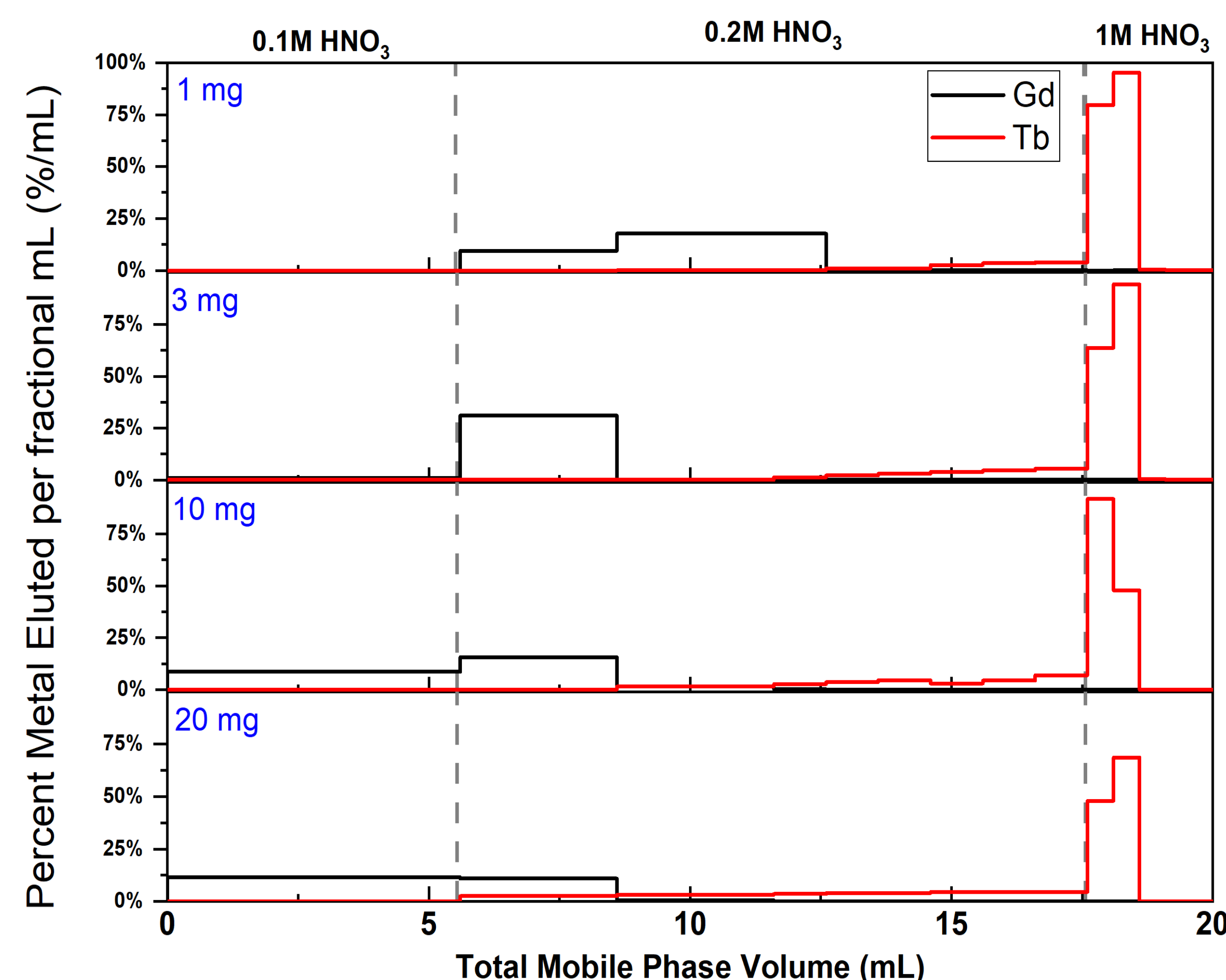


Figure 1: Elution profiles of different columns.

- For columns loaded with 1 mg Gd/Tb, no Gd or Tb can be seen passing through the column prior in the loading solution
- As column size increases to 20 mg, the Terbium peak shrinks and the Gadolinium peak is observably less defined
- In the 10 mg and 20 mg columns, more Terbium elutes earlier, resulting in more overlapping of Gd and Tb peaks



## Gadolinium Recycling

### Motivations

- Production of no-carrier-added Tb-161 would require the use of enriched Gd-160, which is expensive. A recycling process of target material from  $\text{Gd}(\text{NO}_3)_3$  to  $\text{Gd}_2\text{O}_3$  would be required to optimize the Tb-161 production process.

### Methods

- The load and rinse fractions of an LN2 column loaded with 19.5 mg Gd were collected and dried with argon gas (60 mL/min) at 130 C yielding  $\text{Gd}(\text{NO}_3)_3 \cdot 6\text{H}_2\text{O}$ .
- Residue was heated to 400 C to decompose the nitrate to oxide ( $\text{Gd}_2\text{O}_3$ ).
- Masses of the recovered Gd were measured at various steps and compared with expected mass loaded onto the LN2 column.
- This trial has promising indications for Gd target recycling

	$\text{Gd}(\text{NO}_3)_3 \cdot 6\text{H}_2\text{O}$	$\text{Gd}_2\text{O}_3$
Theoretical Yield (mg)	45.94	21.15
Actual Yield (mg)	48.15	20.7
Percent Yield	105%	98%

Figure 3: Yields of different forms of recycled Gd

## Separation Process

### Resin

- 20-50  $\mu\text{m}$  bead size LN2 extraction chromatography resin; dry packed into 300 mg column and mobile phase was flowed through column at 1 mL per minute

### Loading Mass Determination

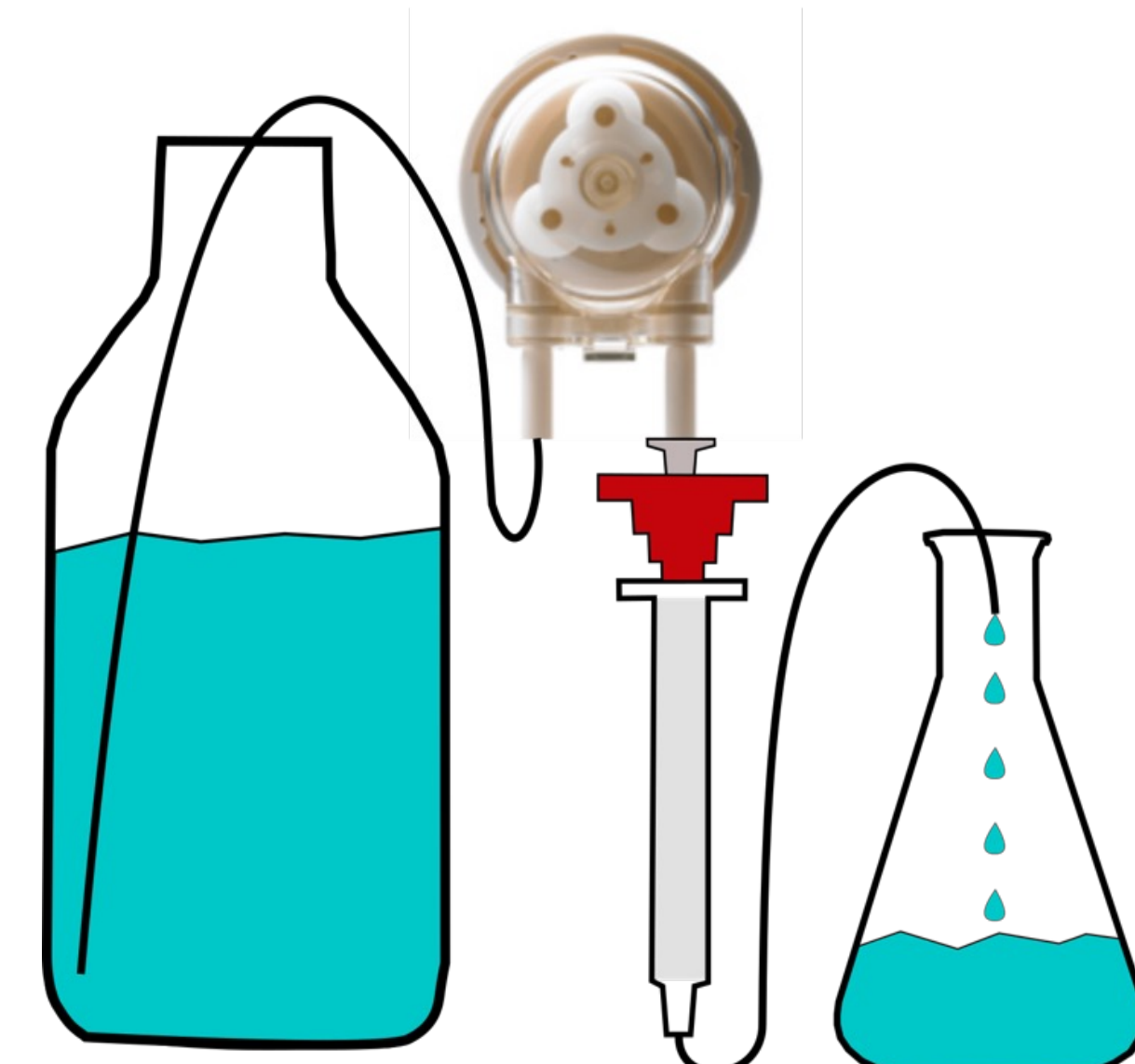
- All trials utilized 500  $\mu\text{g}$  Tb
- Gd mass was tested at 500  $\mu\text{g}$ , 2.5 mg, 9.5 mg, and 19.5 mg

### Mobile Phase Volume Optimizations

- Prep:** 3 mL of 1 M  $\text{HNO}_3$  followed by 14 mL of 0.1M  $\text{HNO}_3$
- Loading:** 5.6 mL of 0.1 M  $\text{HNO}_3$ . Gd and Tb standards were dried down to fit into this mass
- Rinse:** 12 mL of 0.2 M  $\text{HNO}_3$  collected in fractions of 1–3 mL fractions
- Elute:** 3 mL of 1 M  $\text{HNO}_3$  collected in 0.5 mL fractions.

### Analysis

- All trace-metal analysis was performed on Agilent’s microwave plasma atomic emission spectrometer (MP-AES)



30 mm x 5.5 mm  $\varnothing$  column  
300 mg LN2, 20-50  $\mu\text{m}$

Figure 2: A representation of the LN2 column and peristaltic pump used in all separation trials. Liquid was flowed through at 1 mL/min

Gd/Tb Separation Factor equation:

$$Sf_{\text{Gd/Tb}} = \frac{\text{Gd Before}}{\text{Gd After}} \cdot \frac{\text{Tb Before}}{\text{Tb After}}$$

## Results and Discussion

Loaded Gd/Tb mass	Gd/Tb Separation Factor (%Tb recovery)		
	9 mL of 0.2 M $\text{HNO}_3$	10 mL of 0.2 M $\text{HNO}_3$	12 mL of 0.2 M $\text{HNO}_3$
1 mg	900 (98%)	2000 (95%)	6100 (88%)
3 mg	700 (95%)	900 (91%)	1100 (82%)
10 mg	1900 (84%)	2700 (81%)	10000 (70 %)
20 mg	1300 (72%)	1400 (68%)	1500 (58%)

Table 1: Separation factor and terbium percent recovery calculated after 9 mL rinse, 10 mL rinse, and 12 mL rinse.

Objective: Maintaining both a high Gd/Tb separation factor (>1000) and a high Tb % recovery (>90%)

- Tb percent recovery drops as Gd mass increases
  - Separation factor for a given Tb recovery drops as Gd mass increases
  - Separation must optimize both of these factors
- However ...
- The 300 mg LN2 column still adequately separated Tb and Gd up to 20 mg total (~90% of its theoretical capacity)
  - Eluting Terbium between 9 and 12 mL 0.2 M  $\text{HNO}_3$  in all trials maintains the highest Sf and % recovery

## Future Production

Calculations based on literature yields [3] were made to determine theoretical yield of Tb-161 from enriched Gd-160 targets at the UW-Madison Nuclear Reactor Facility.

### UW Nuclear Reactor Considerations

- $1.1 \times 10^{13}$  n/cm<sup>2</sup>/s neutron flux
- Possible irradiation time – any lengths over 1 day are likely not feasible. Due to time constraints, the Physical Yield of 5.5  $\mu\text{Ci}/\text{mg}/\text{h}$  is more applicable than the Saturation Yield of 1.3 mCi/mg.

### Theoretical Yield

- A 4-hour irradiation of 100 mg  $^{160}\text{Gd}_2\text{O}_3$  would produce 2.2 mCi  $^{161}\text{Tb}$ 
  - Sufficient for *in vitro*  $^{161}\text{Tb}$ -based radiopharmaceutical evaluation

## References

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